

# Fabrication and Characterization of Uranium-based Nuclear Fuels

# Background and Introduction

which was confirmed through X-ray diffraction.

optimization of high performance, accident tolerant fuels.



Physical Properties	UN	UO <sub>2</sub>
Density (g/cm <sup>3</sup> )	14.33	10.97
Uranium Density (g/cm <sup>3</sup> )	13.53	9.67
Melting Point (°C)	2650	2700
Heat Capacity (J/kgK) at 500°C	230	300
Thermal Conductivity (W/mK) at 500 °C	20 5	2_/

Table 1: Bulk physical properties of UN and UO<sub>2</sub>. The high uranium density, thermal conductivity, and low heat capacity of UN are favorable over UO<sub>2</sub> as a nuclear fuel [3].









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d Pellet Density	UN	UO <sub>2</sub>	
retical Density (g/cm <sup>3</sup> ) [3]	14.33	10.97	
Density (% of theoretical)	60 ± 2	55 ± 5	
Density of the pollets ofter being pressed into shape			

## Acknowledgements

UN: High purity, 9 mm diameter pellets were fabricated with greater than 90% theoretical density.

Ultra-high-purity argon atmosphere formed pellets with the less porosity than those sintered in Ar + 100 ppm N<sub>2</sub>. Further analysis such as elemental analysis and further sintering studies need to be carried out for a complete understanding.



Figure 9: Scanning electron micrographs of UN pellets sintered in a.) ultra-highpurity argon, and b.) ultra-high-purity argon + 100 ppm  $N_2$ . Undesirable porosity resulted when N<sub>2</sub> was in the sintering atmosphere.

**UO<sub>2</sub>:** The sintering process effectively removed the  $U_3O_8$ impurity in the starting powder. This occurs during the first and third stages of sintering (see Figure 8a) where the reducing environment causes the following reaction to take place [6]:

$$U_3O_8 + 2H_2 \rightarrow 2H_2O +$$

The resulting microstructure of the 3 mm pellet was small pores (<1um) that are spherical in shape. The grain size could not be determined without additional etching. A similar process will be used to fabricate 9 mm pellets as the project continues.

## % Theoretical Density of Pellets Archimedes Density

Table 3: Final densities of the UN and  $UO_2$  pellets. Error for Archimedes density was determined through error analysis, after three sequences of weighing.

## Conclusions

Carbothermic reduction is a suitable route for preparing UN from UO<sub>2</sub>. High purity, dense pellets of UN and UO<sub>2</sub> nuclear fuels were successfully made. For UN sintering, nitrogen free atmosphere results in less porosity. This project demonstrates the feasibility to fabricate phase pure UN and UO<sub>2</sub> nuclear fuel pellets for further studies.

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## $+ 3UO_{2}$ eq. 4





ray diffraction patterns (c) of sintered UN pellets. There is significant UO<sub>2</sub> impurity present in the 3 mm pellets, but surface grinding of the 9 mm pellet produced an XRD pattern with no detectable second phase



Figure 11: X-ray diffraction pattern of sintered 3 mm UO<sub>2</sub> pellet. Although  $U_3O_8$  was detected in the starting powder, the sintering process effectively removed this impurity.



Figure 12: a.) Scanning electron micrograph revealing fine pore structure. b.) Photograph of the UO<sub>2</sub> pellet.





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